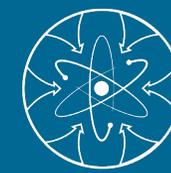


# Reactor Power Synthesis with Simulated SPND Responses & SPND Analysis of WIRE-21



# Reactor Power Synthesis with Simulated SPND Responses

**Advanced Sensors and Instrumentation (ASI)  
Annual Program Webinar**

October 30 – November 2, 2023

Anthony Birri, K. C. Goetz,

Daniel C. Sweeney, Tyler Gates, N. Dianne Bull Ezell

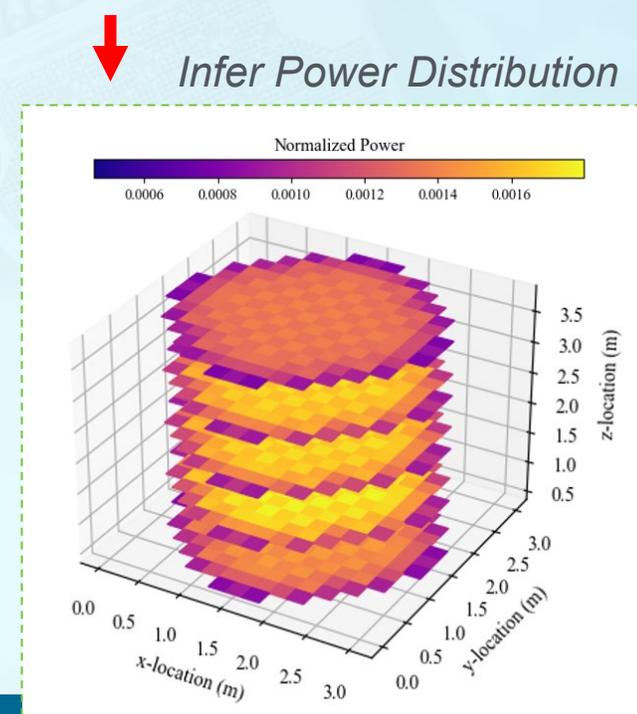
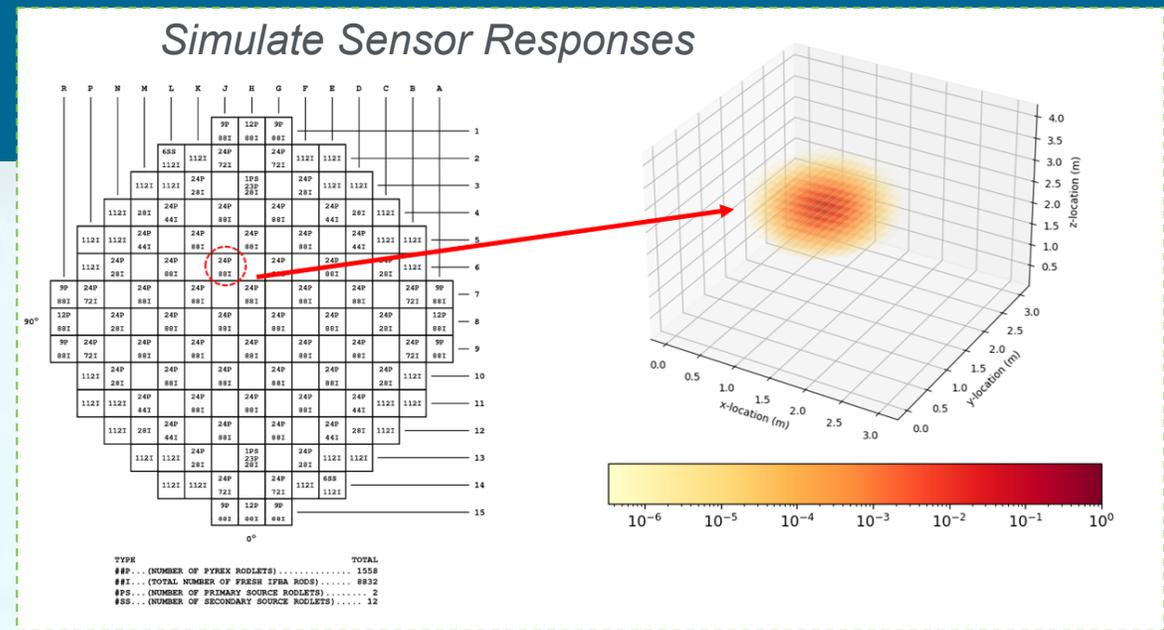
**Oak Ridge National Laboratory**

# Project Overview

- The goal of this project is to utilize a weighting function based method to synthesize core power based on simulated sensor responses
  - Reactor models come from MCNP, sensor physics can come from Geant4
- Looking at multiple reactors in order to assess
  - Impact of sensor uncertainty
  - What types of perturbations can be accurately detected
  - Sensor arrangement optimization
- In FY23, there were to main focuses:
  1. **Assess impact of sensor uncertainty in AP1000 and NuScale SMR**
  2. **Develop a highly realistic model and demonstrate perturbation detection w/ TAMU TRIGA**
- Participants
 

ORNL: Anthony Birri (PI), Callie Goetz (Geant4 modeling), Daniel Sweeney (python expertise), N. Dianne Bull Ezell (supervision)

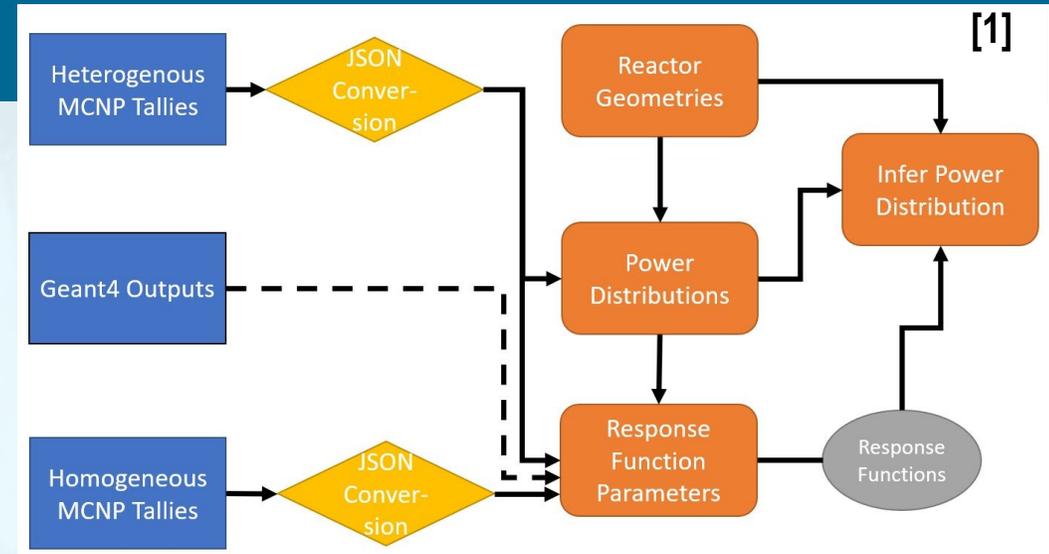
TAMU: Tyler Gates (TAMU, MCNP modeling)



# Overview of Methodology

- The method we have developed is coined the “Point Based Iterative” (PBI) method
  - It is fundamentally a weighting function based method
- The idea is that each sensor can provide an estimate of the power in each ‘chunk’ of fuel via a response function
  - MCNP informed flux data
- Through a weighted average, each chunk power is determined based on all the sensor estimates
  - This is an iterative process

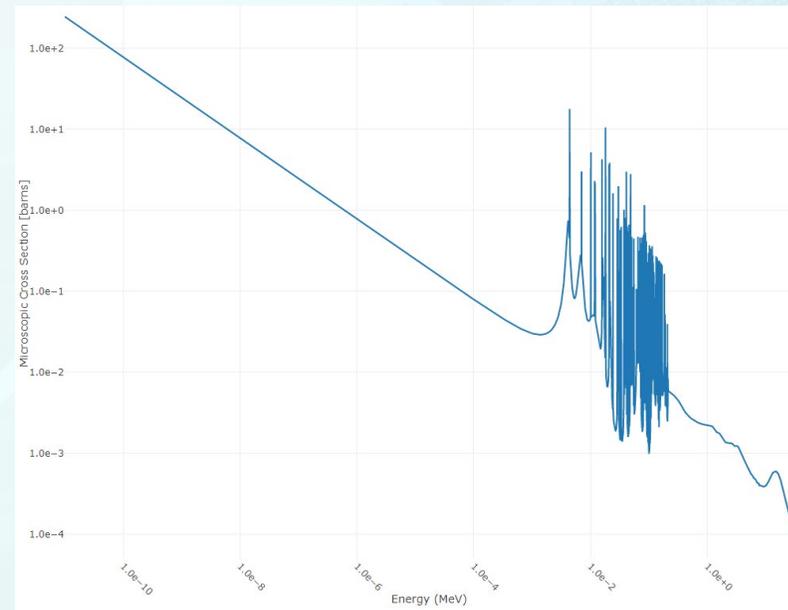
## Software Flowchart



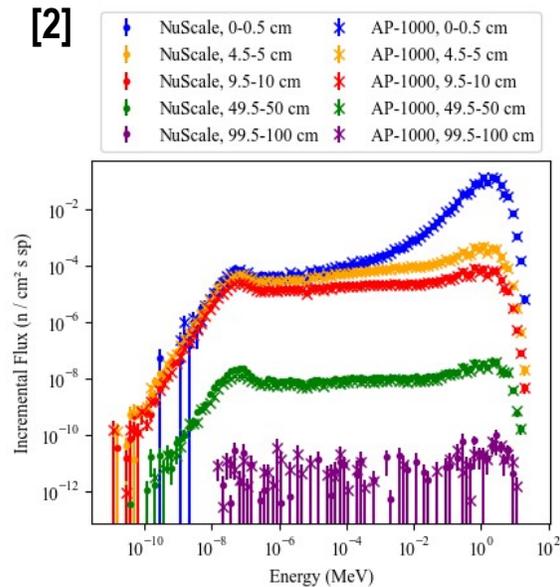
[1] DOI:  
10.2172/1996662

[2] DOI:  
10.1016/j.pnucene.  
2022.104437

## V-51 neutron absorption cross section



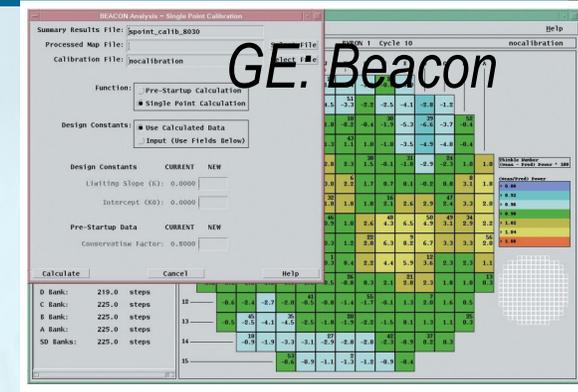
## Flux Characterization



# Technology Impact

- It is clear that core power shape synthesis is utilized by industry
- However, efficacy of implemented methods in software is unclear
- A small body of research literature exists, but there are still many questions which remain, regarding optimization, uncertainty, etc.

Industry Software



**ACUMEN™**  
Next Generation Core Monitoring System

ACUMEN™ is a complete fuel monitoring system, designed by GNF, to have the same functionality as the existing 3DMONICORE BWR core monitoring and predicting system, along with new, enhanced capabilities

**GNF**  
Global Nuclear Fuel  
A joint venture of GE Hitachi & Framatome

BWR Excellence through Innovation

## Patents

**United States Patent** [19] [11] Patent Number: **5,490,184**  
Heibel [45] Date of Patent: Feb. 6, 1996

[54] METHOD AND A SYSTEM FOR ACCURATELY CALCULATING PWR POWER FROM EXCORE DETECTOR CURRENTS CORRECTED FOR CHANGES IN 3-D POWER DISTRIBUTION AND COOLANT DENSITY

[75] Inventor: Michael D. Heibel, Penn Township, Pa.

[73] Assignee: Westinghouse Electric Corporation, Pittsburgh, Pa.

[21] Appl. No.: 278,290

[22] Filed: Jul. 21, 1994

[1] Int. Cl.<sup>6</sup> ..... G21C 17/00

[52] U.S. Cl. .... 376/254; 376/247; 376/246; 376/258; 376/216

[58] Field of Search ..... 376/254, 255, 376/247, 246, 258, 241, 216; 916/DIG. 238, DIG. 239; 222/54; 374/30, 112, 127, 174

**United States Patent** [19] [11] Patent Number: **4,637,910**  
Impink, Jr. [45] Date of Patent: Jan. 20, 1987

[54] METHOD AND APPARATUS FOR CONTINUOUS ON-LINE SYNTHESIS OF POWER DISTRIBUTION IN A NUCLEAR REACTOR CORE

[75] Inventor: Albert J. Impink, Jr., Murrysville, Pa.

[73] Assignee: Westinghouse Electric Corp., Pittsburgh, Pa.

[21] Appl. No.: 572,499

[22] Filed: Jan. 20, 1984

[51] Int. Cl.<sup>4</sup> ..... G21C 7/30

[52] U.S. Cl. .... 376/216; 217, 215

[58] Field of Search ..... 376/216, 217, 215

[56] References Cited  
U.S. PATENT DOCUMENTS  
4,066,497 1/1978 Sato ..... 376/211  
4,075,059 2/1978 Bruno ..... 376/211  
4,218,778 3/1982 Musick ..... 376/214  
4,432,930 2/1984 Impink ..... 376/211

Primary Examiner—Donald P. Walsh  
Attorney, Agent, or Firm—Daniel C. Abeles

**ABSTRACT**  
On-line, real time monitoring of core-wide and local

13 Claims, 5 Drawing Sheets

**United States Patent** [19] [11] Patent Number: **4,333,797**  
Nishizawa [45] Date of Patent: Jun. 8, 1982

[54] REACTOR POWER CONTROL APPARATUS

[75] Inventor: Yasuo Nishizawa, Hitachi, Japan

[73] Assignee: Hitachi, Ltd., Tokyo, Japan

[21] Appl. No.: 147,077

[22] Filed: May 7, 1980

[30] Foreign Application Priority Data  
May 11, 1979 [JP] Japan ..... 54-57045

[51] Int. Cl.<sup>3</sup> ..... G21C 7/00

[52] U.S. Cl. .... 376/210; 376/216; 376/217

[58] Field of Search ..... 176/20 R, 22, 24

[56] References Cited  
U.S. PATENT DOCUMENTS  
4,108,720 8/1978 Sato et al. .... 176/24  
4,236,220 11/1980 Kogami et al. .... 176/20 R

Primary Examiner—Sal Cangialosi  
Attorney, Agent, or Firm—Craig and Antonelli

**ABSTRACT**  
A control apparatus for a boiling water reactor includes a monitoring section for monitoring power distribution

3 Claims, 10 Drawing Figures

## Existing Studies

Annals of Nuclear Energy  
Volume 72, October 2014, Pages 467-470

Reactor power shape synthesis using Group Method of Data Handling

Moon-Ghu Park, Ho-Cheol Shin

Progress in Nuclear Energy  
Volume 131, January 2021, 103574

Reconstruction of neutron flux distribution by nodal synthesis method using online in-core neutron detector readings

Iman Ramezani, Mohammad Bagher Ghofrani

Progress in Nuclear Energy  
Volume 153, November 2022, 104437

Simulating self-powered neutron detector responses to infer burnup-induced power distribution perturbations in next-generation light water reactors ☆

Anthony Birri, Daniel C. Sweeney, N. Dianne Bull Ezell

Progress in Nuclear Energy  
Volume 31, Issue 4, 1997, Pages 369-372

Harmonics synthesis method for core flux distribution reconstruction

Li Fu, Luo Zhengpei, Hu Yongming

# Developments to Date

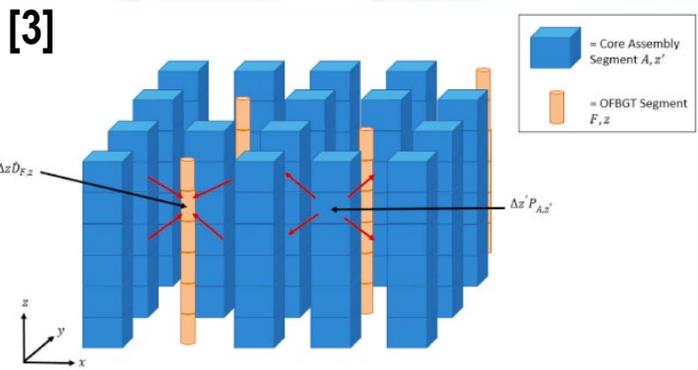
- The current project stems from previous work with OFBGT development at OSU
  - A method was developed to synthesize core power based off of an OFBGT array
- This method was demonstrated experimentally in the OSURR
  - Reasonable agreement, sensor design could be improved
- The method was adapted to intake SPND data at ORNL
  - A study was conducted to assess core follow impact on a variety of sensor-core configurations
  - Studied in the context of AP1000 and NuScale SMR

[2] DOI: 10.1016/j.pnucene.2022.104437

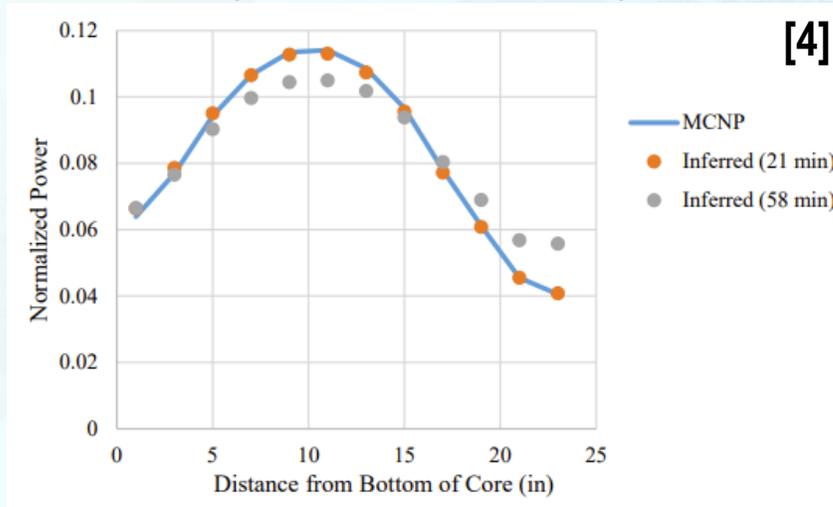
[3] DOI: 10.1016/j.pnucene.2020.103552

[4] Birri, Dissertation (2021)

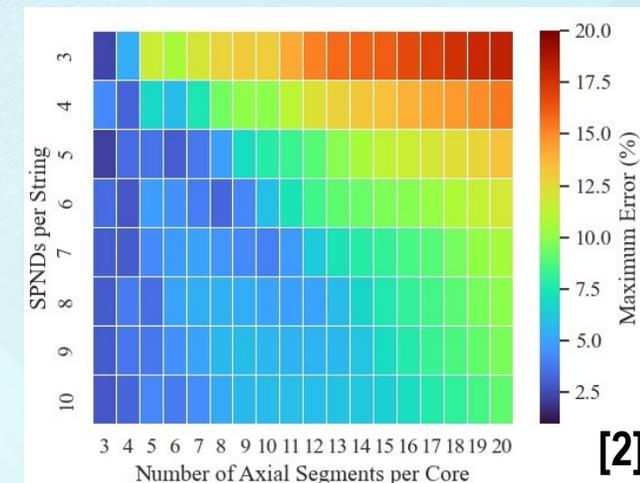
Methodology Development  
(for OFBGTs, 2018-2020)



Experimental Application  
(with OFBGTs, 2020-2021)

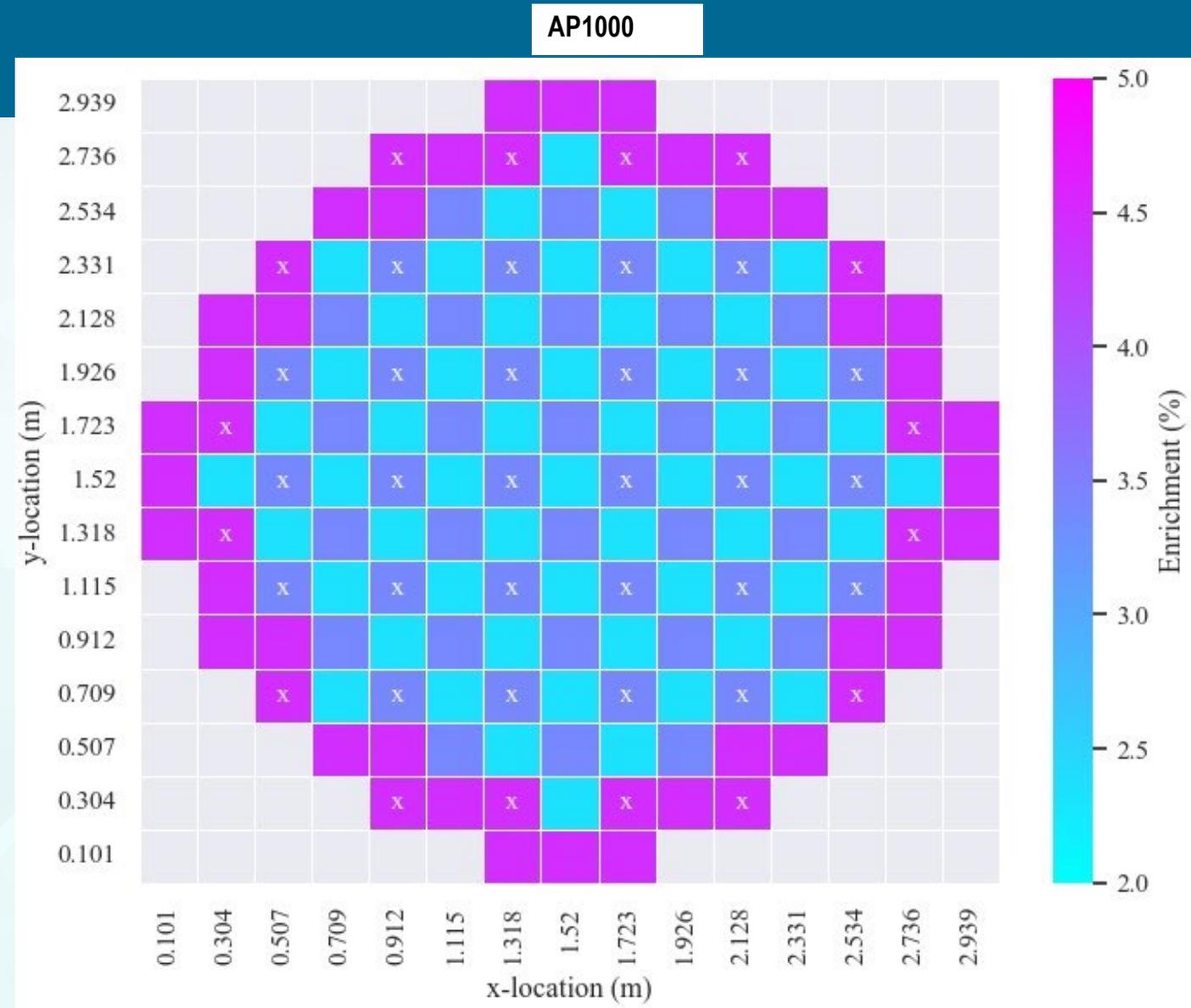
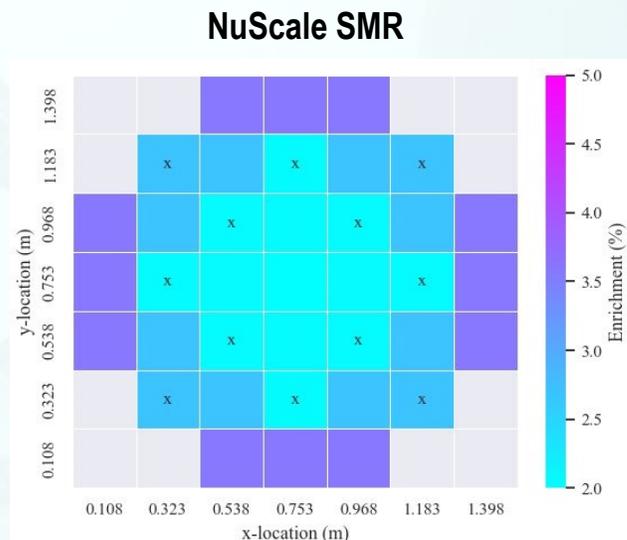


Adaptation to SPNDs, and further  
analysis (2021-present)



# Next Gen LWR Model Details

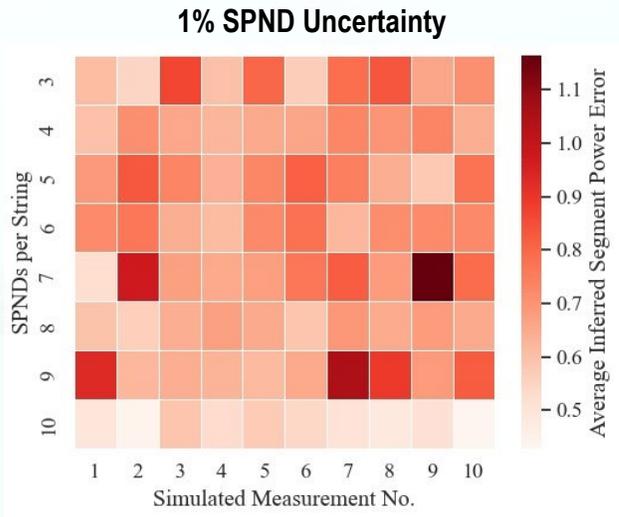
- NuScale SMR and AP1000 are pressurized LWRs
- They both use SPNDs, assumedly Vanadium emitters
- Assumed power distributions determined heterogeneously
- Response functions determined homogeneously (i.e. less realistic sensor responses)



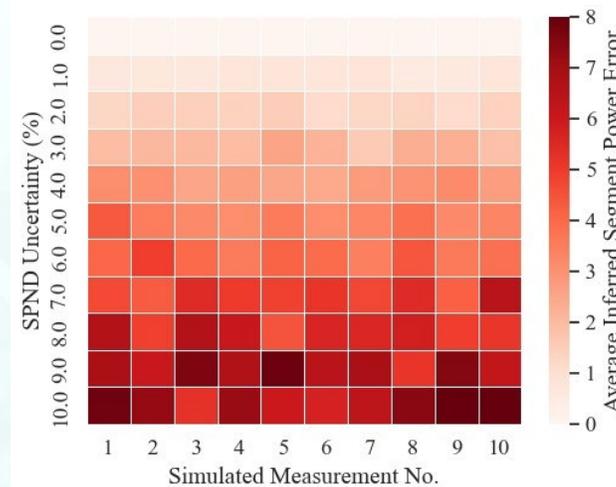
# Next Gen LWR Uncertainty Analysis Results

- Varied SPND uncertainty and number of SPNDs per string, assessed error in power synthesis
  - Increasing uncertainty results in increased average error (not a surprise), close to a 1:1 trend
  - Minimal benefit to increasing number of SPNDs per string from 3 to 10
- AP1000 slightly more prone to error for same SPND uncertainties

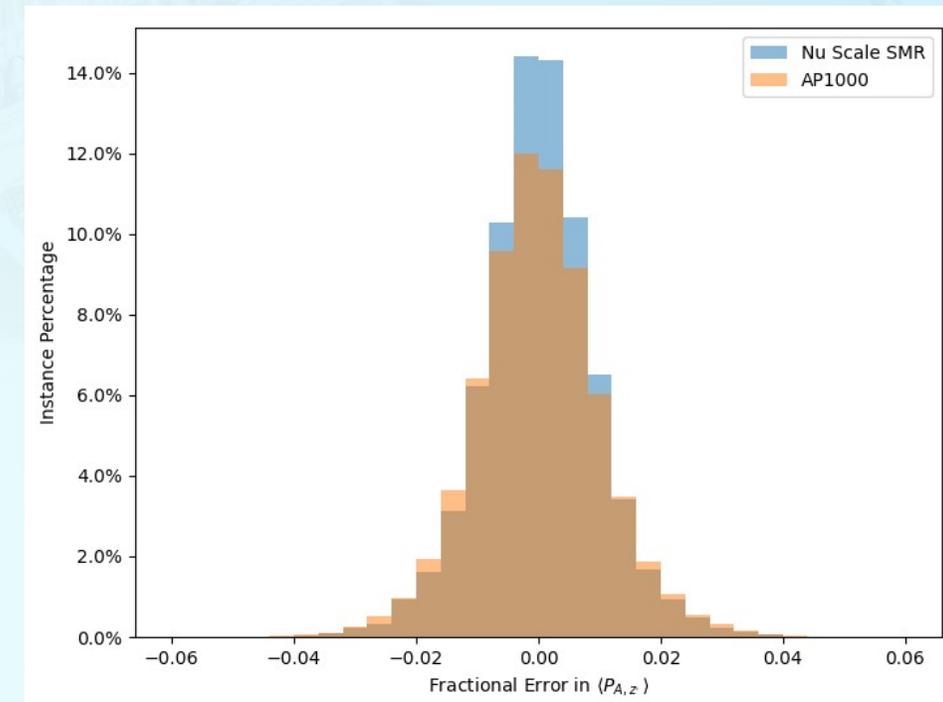
## NuScale Error Results



## 4 SPNDs per string

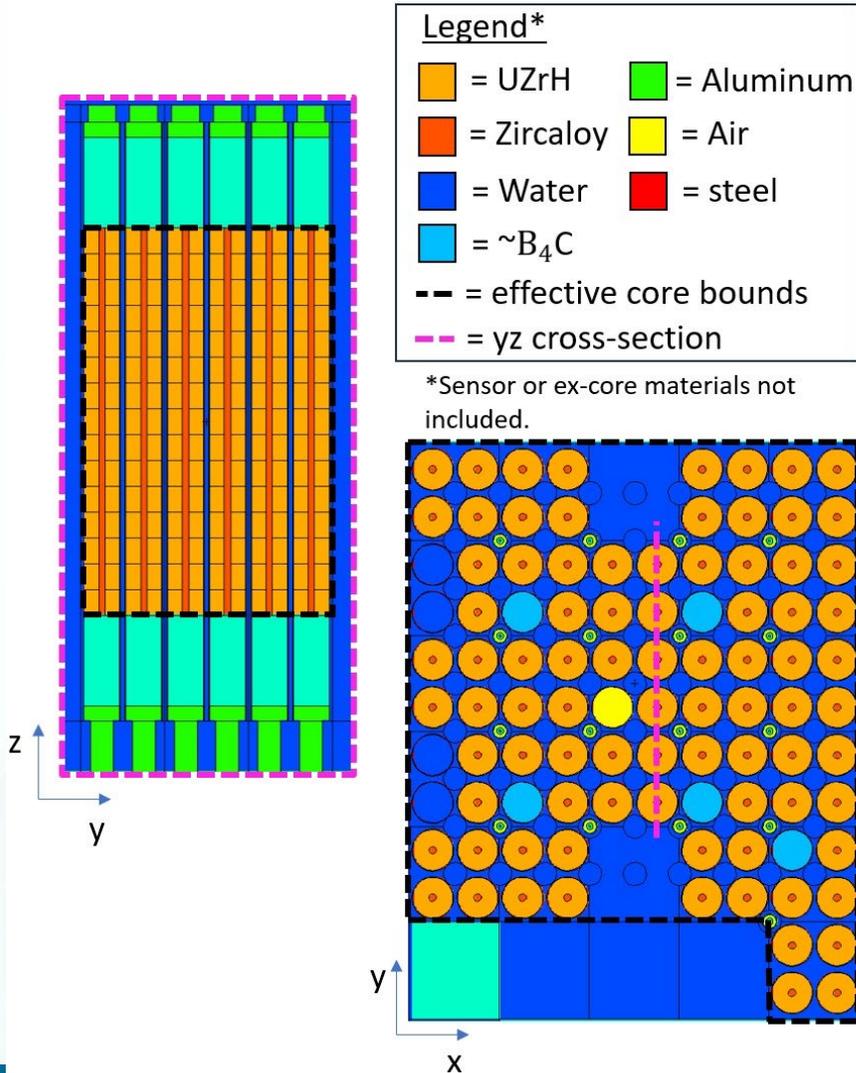


## Distribution of Error: Reactor Comparison w/ 1% SPND uncertainty

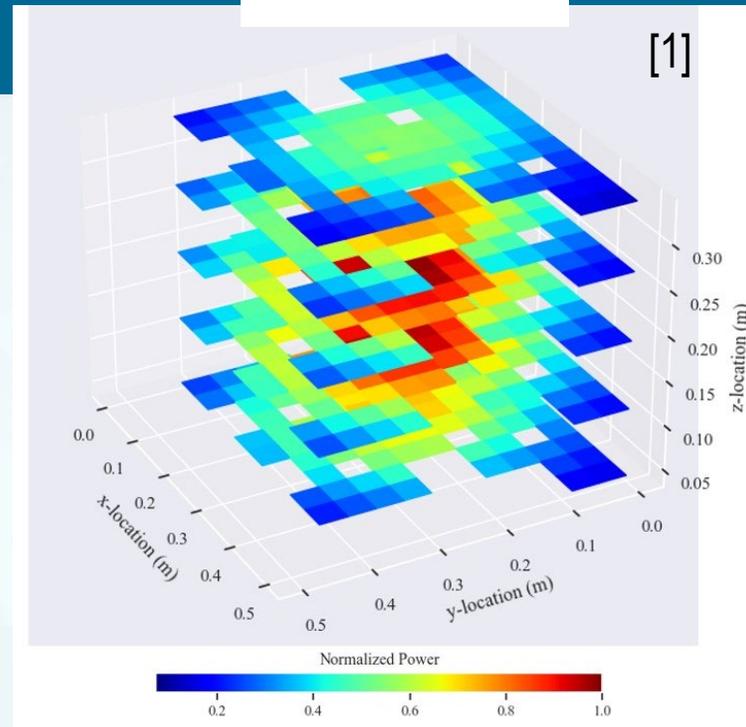


# TAMU TRIGA Model Details

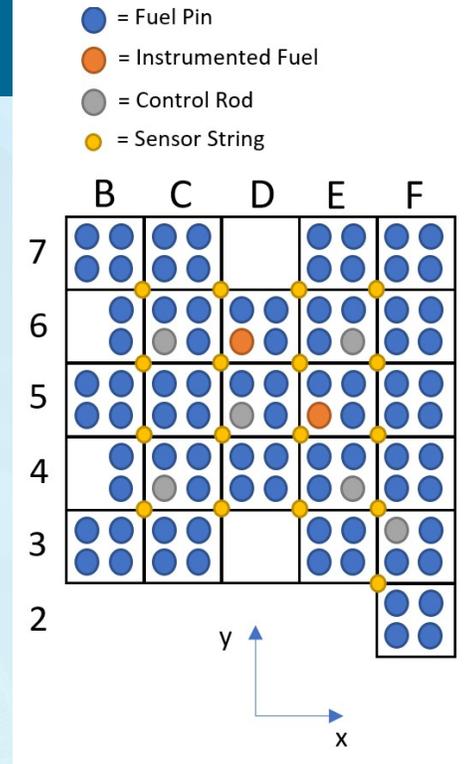
## MCNP Model



## Power Distribution



## Simple Schematic

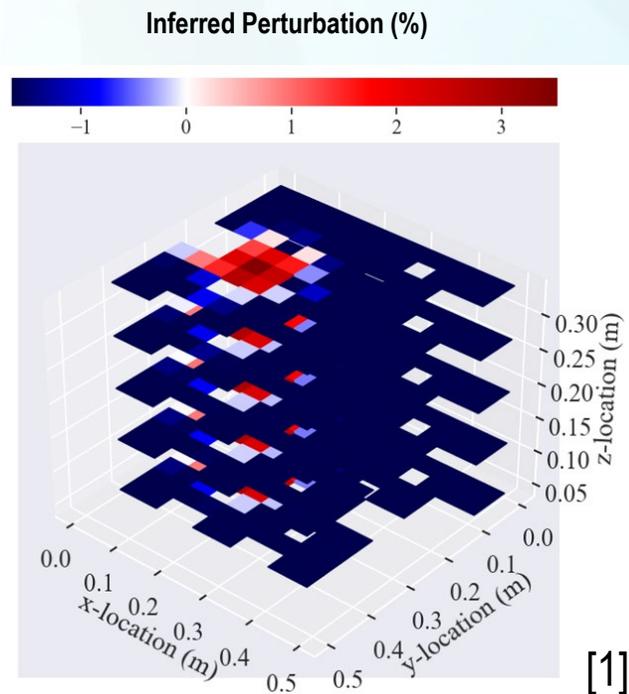
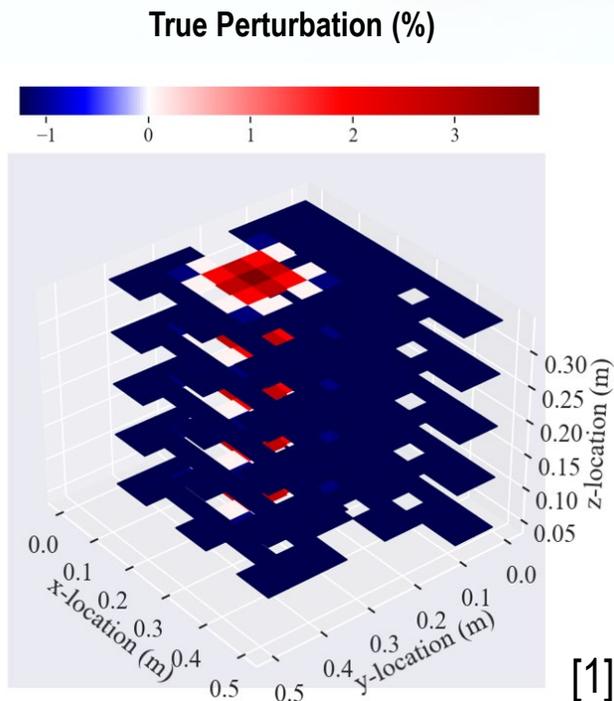


- Pool-type research reactor w/ TRIGA elements (UZrH fuel)
- 17 modeled string locations, 4 SPNDs assumed per string
- Heterogeneous power distribution and response function calculations (more realistic)

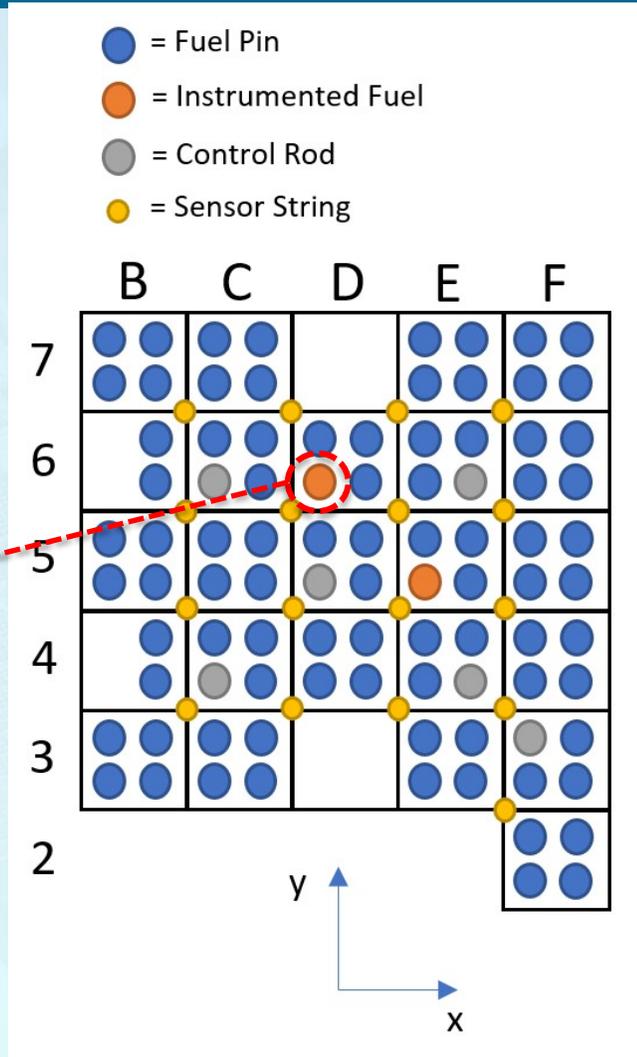
[1] DOI: 10.2172/1996662

# TAMU TRIGA Perturbation Analysis Results

- Considered a Gaussian type Perturbation centered on instrumented fuel pin
  - Variance of Gaussian was 0.125 m
- Average error in synthesized distribution was 0.19%, general shape is clearly accurate
- Note: SPND responses assumed to be responding perfectly



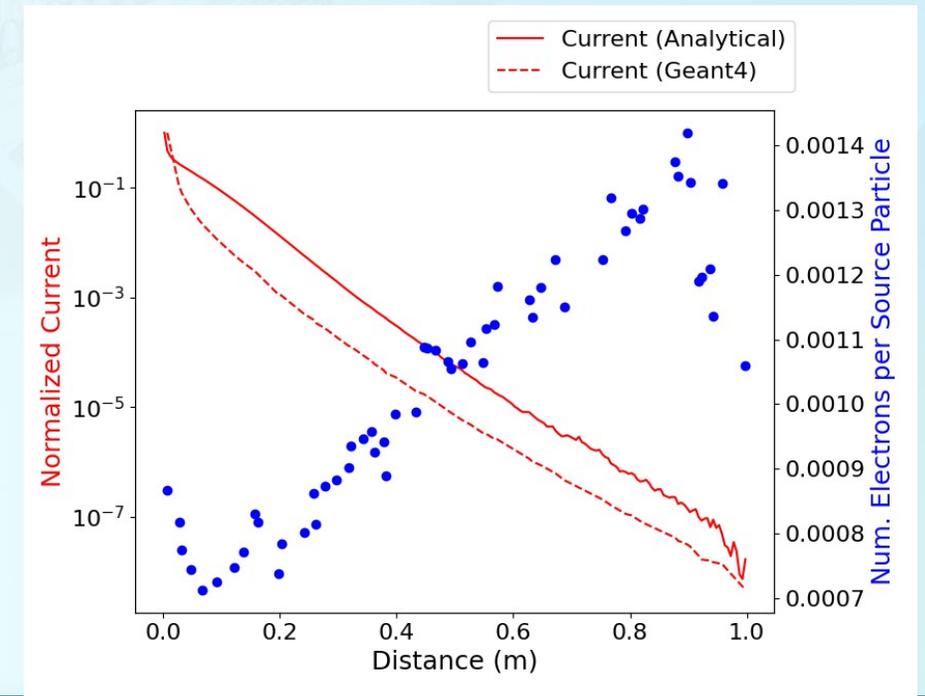
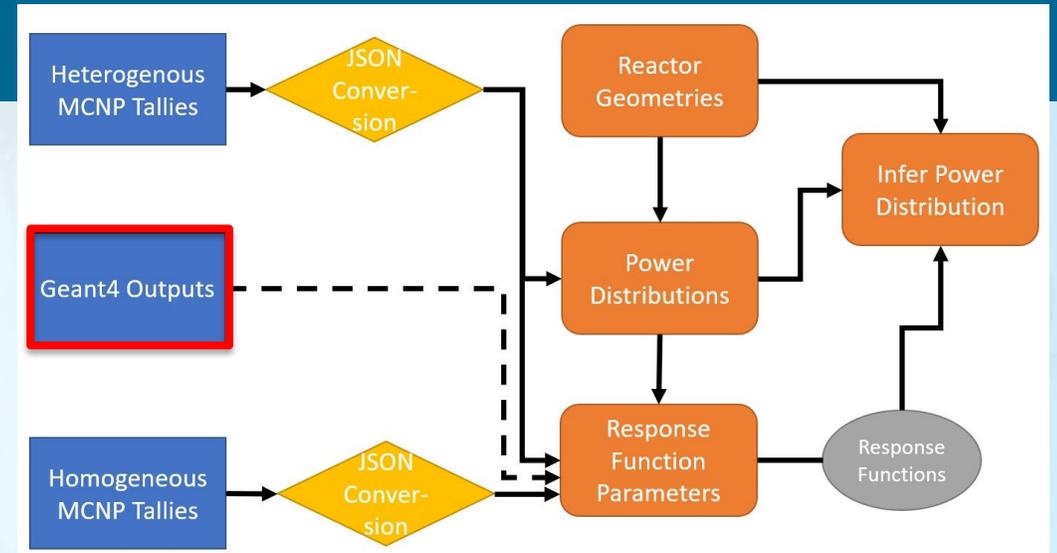
Center of Perturbation



[1] DOI: 10.2172/1996662

# Geant4 Integration

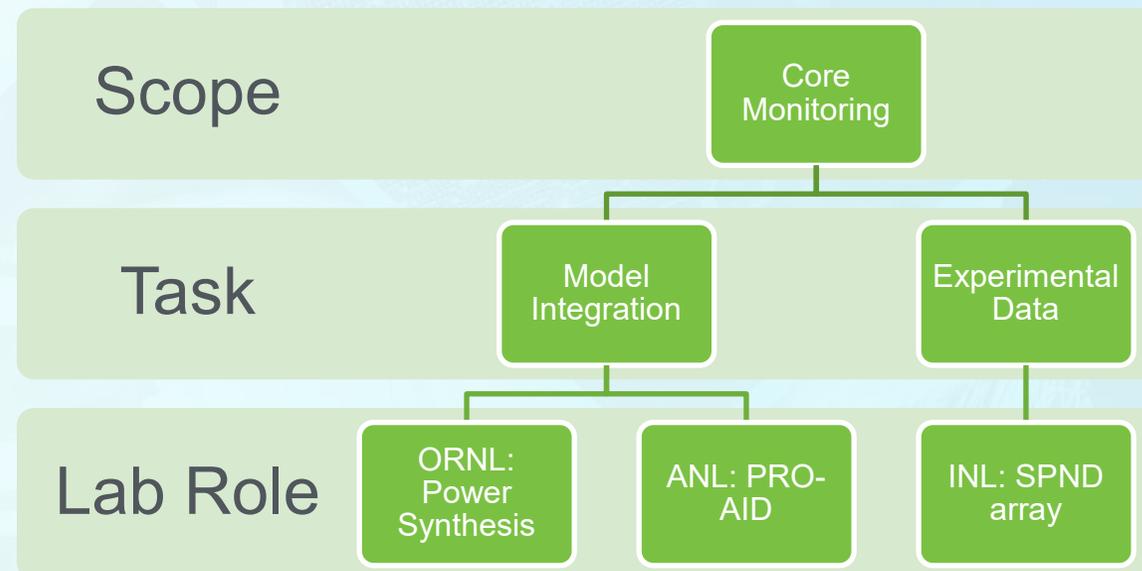
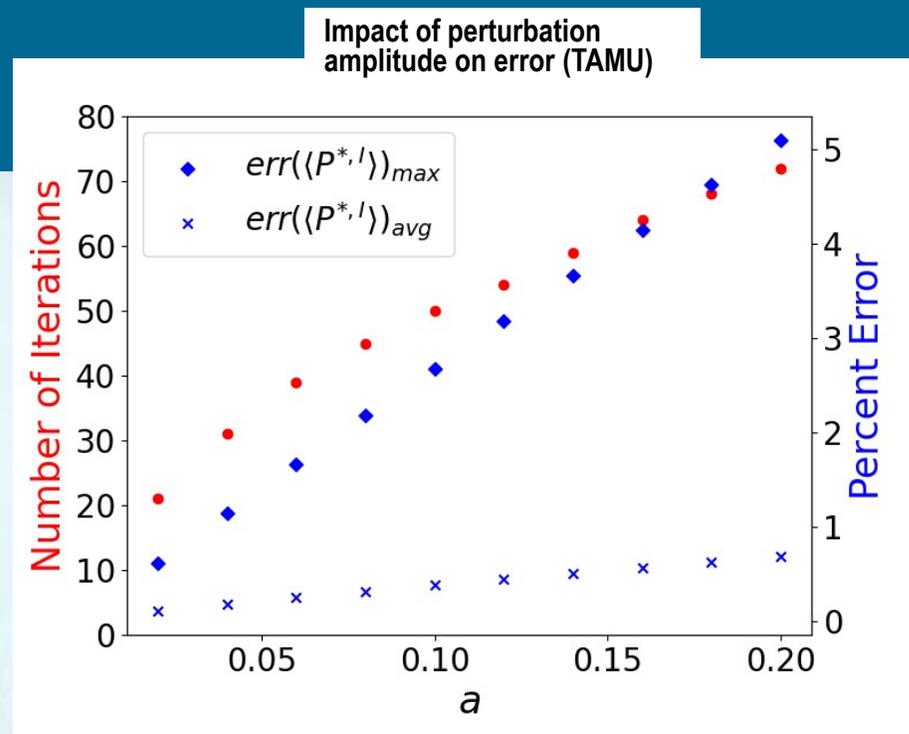
- Developed a Geant4 SPND model to integrate with power synthesis software
  - Monte Carlo sensor modeling package developed by CERN
- Currently, analytical models are utilized
  - Doesn't account for neutron self-shielding
  - Doesn't account for electric field effects in insulator
- Preliminary results w/ NuScale highlight impact of self-shielding on current response
- Currently working on Geant4 model E-field optimization, and software integration



POC for Geant4: Callie Goetz

# Follow-On Work

- Perform additional analyses with TAMU TRIGA to identify trends in perturbation variables on synthesis error
- After integration of Geant4, reassess uncertainty impacts in next gen LWRs
- Collaborate with INL for future experimental collaboration with SPNDs for core power monitoring
- Identify potential for connection between the power synthesis software developed at ORNL and PRO-AID developed at ANL



# Concluding Remarks

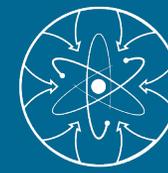
- Power synthesis is a crucial core monitoring capability which reactor operators must have for safe operations
- ORNL is addressing some of the many questions in this scope with targeted studies of uncertainty, perturbations, and sensor arrangement alterations
- AP1000 and NuScale SMR have served as model test beds for sensor uncertainty analysis
- A high fidelity, realistic TAMU TRIGA MCNP model has been used to develop a highly realistic modeling framework, which has been demonstrated with perturbation detection simulation.

## Acknowledgements:

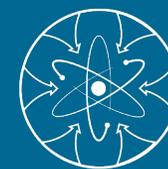
- This work was directly funded by the ASI Program under the U.S. Department of Energy Office of Nuclear Energy
- Credit goes to Thomas Blue at OSU for conceptualization of the methodology used herein

### Anthony Birri

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R&D Associate Staff Member  
Nuclear and Extreme Environment Measurements Group  
Cell: (614) 406-2416 | birriah@ornl.gov



**Thank You**



# SPND Analysis of WIRE-21

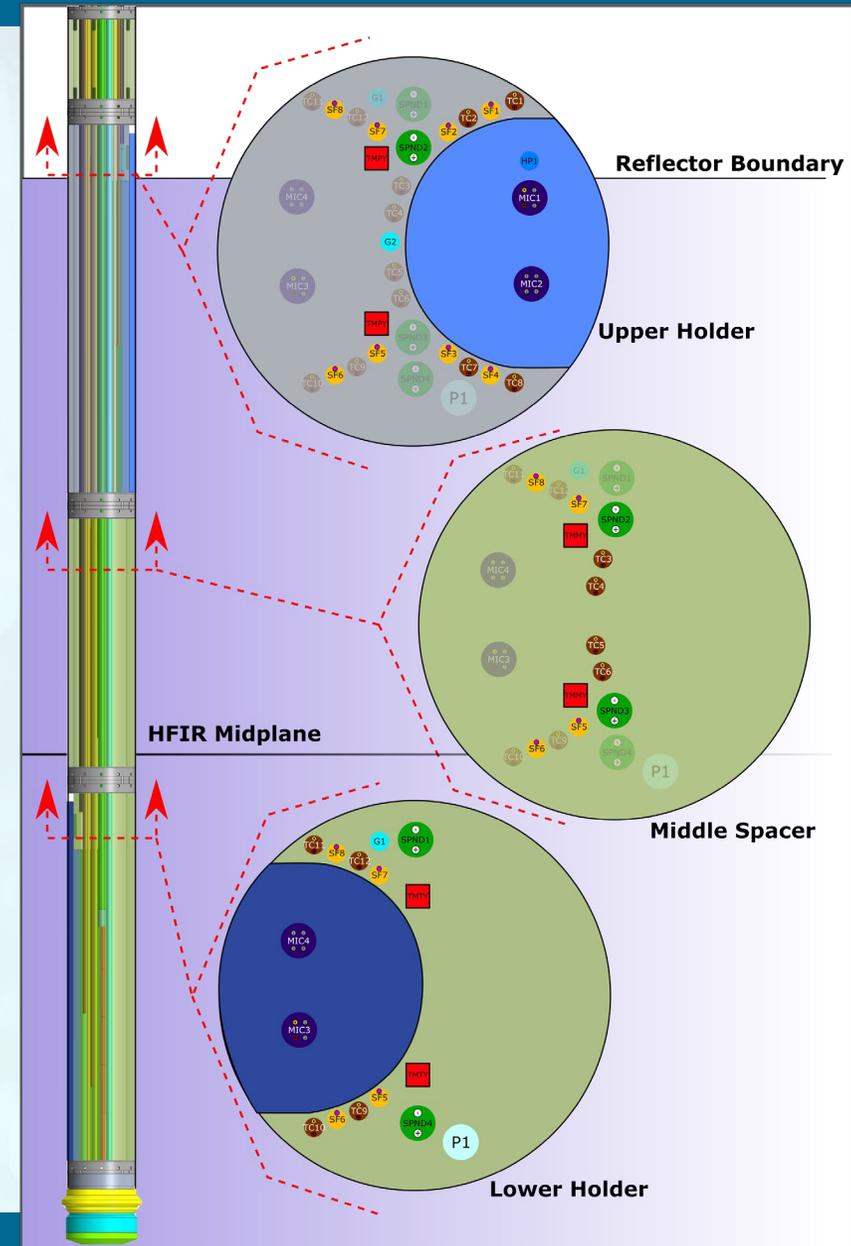
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Oak Ridge National Laboratory

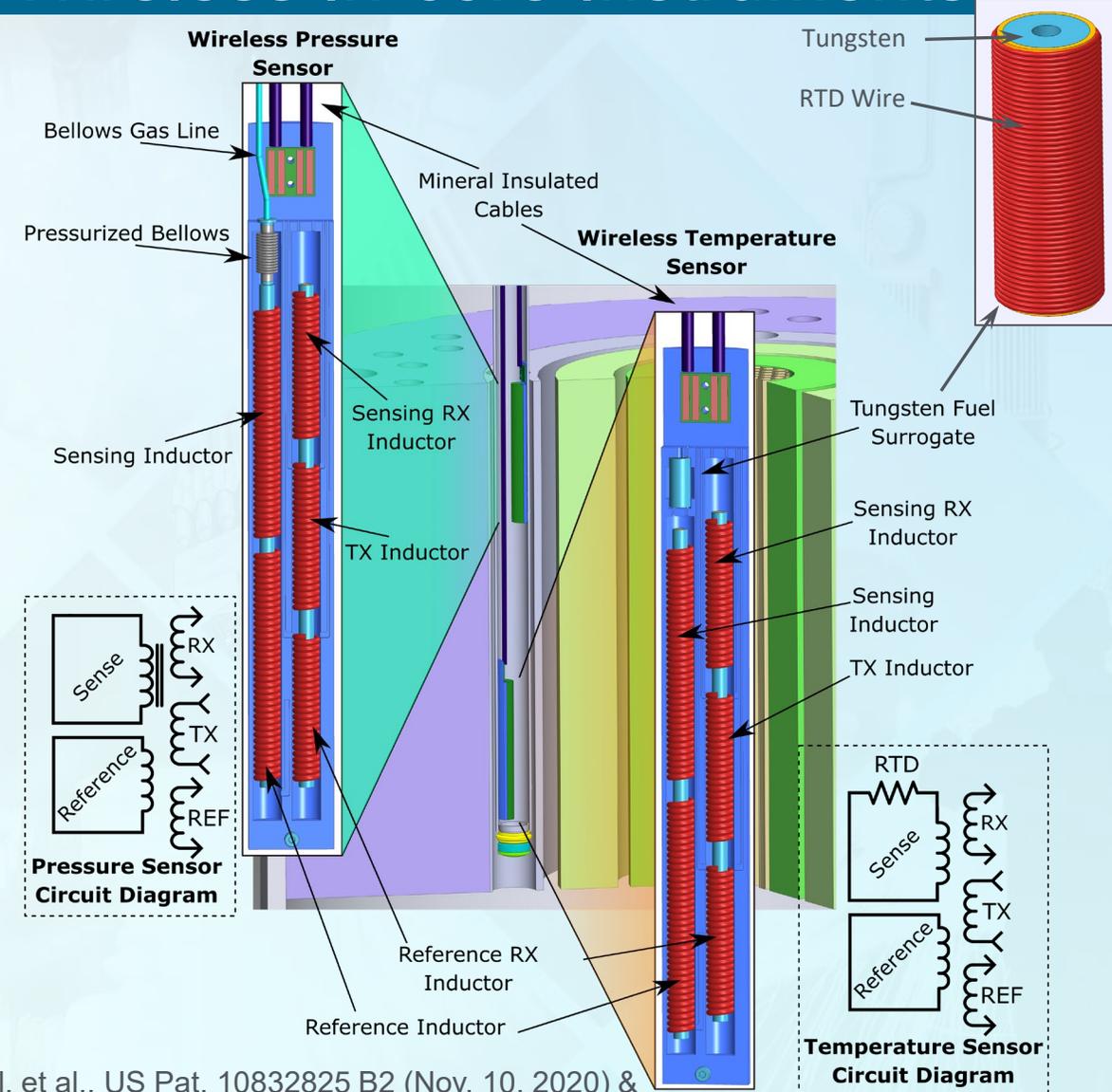
# Wireless Instrumented Removable beryllium Experiment (WIRE-21)

- Most highly instrumented experiment in HFIR's 58-year history
- Designed to test several sensing technologies in real-time
  - Validate instruments for future real-time in-core testing
  - Compare against established technologies
- Three primary zones (holders) for experiment arrangement and heat transfer
  - Active temperature & pressure control
- Primary purpose to test wireless sensors developed by Westinghouse Electric Company (WEC) for 3 HFIR cycles



# Westinghouse Electric Company Wireless In-core Instruments

- WEC wireless pressure sensor [1]
  - Moveable ferritic core connected to bellows
  - Inductive coupling energizes sensing and reference inductors
  - Probed using complimentary receiving inductors
- WEC wireless temperature sensor [1]
  - Tungsten cylinder acts as fuel surrogate material
  - Thermocouple attached for local temperature monitoring
  - Wire wrap operates as resistance temperature detector (RTD)



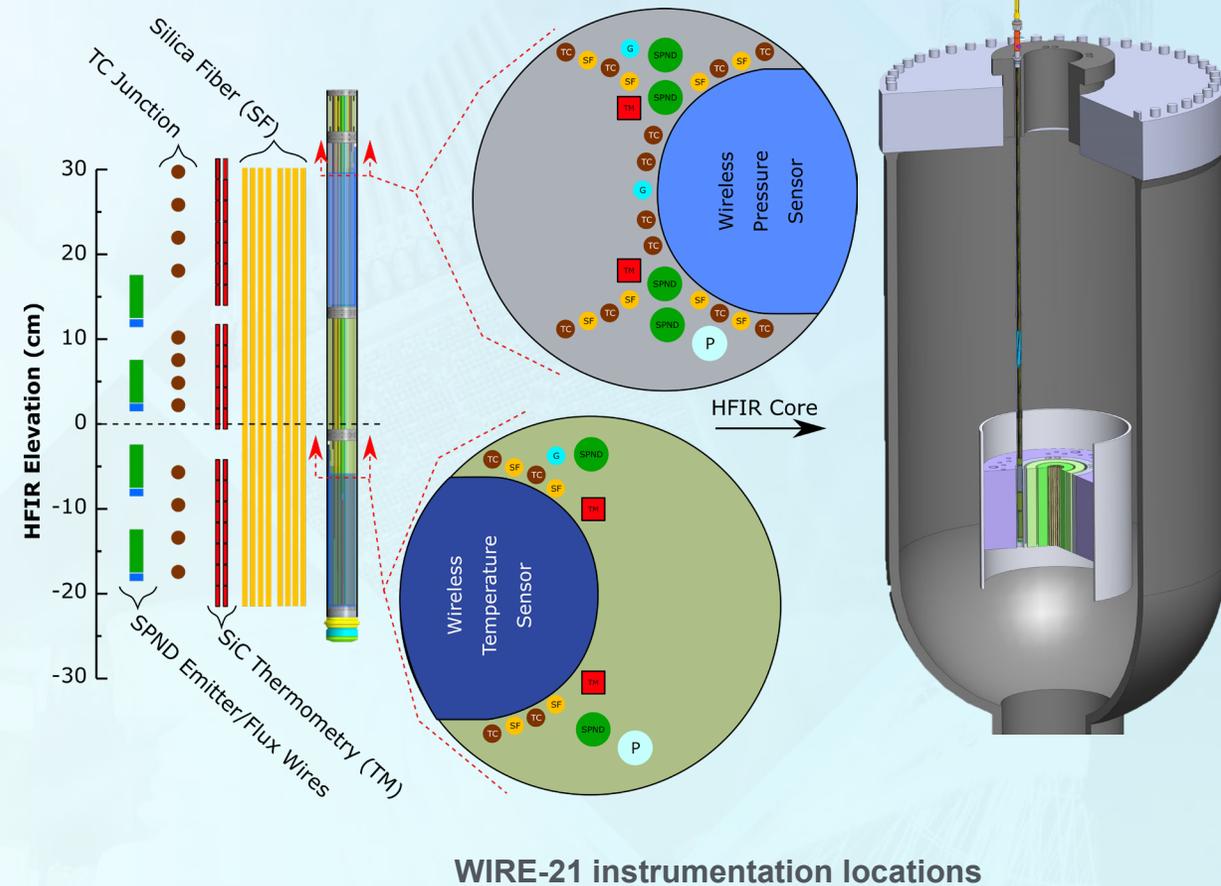
[1] J. Carvajal, et al., US Pat. 10832825 B2 (Nov. 10, 2020) & US Pat. 10811153 B2 (Oct. 20, 2020)

# WIRE-21 Instrumentation

- Temperature, pressure, neutron flux measured in multiple positions along reflector
  - Total of 70 independent measurements
  - 7 measurement techniques
- Collected in real-time and through PIE
- Spatially discrete and continuous measurements
- Range of technological maturity

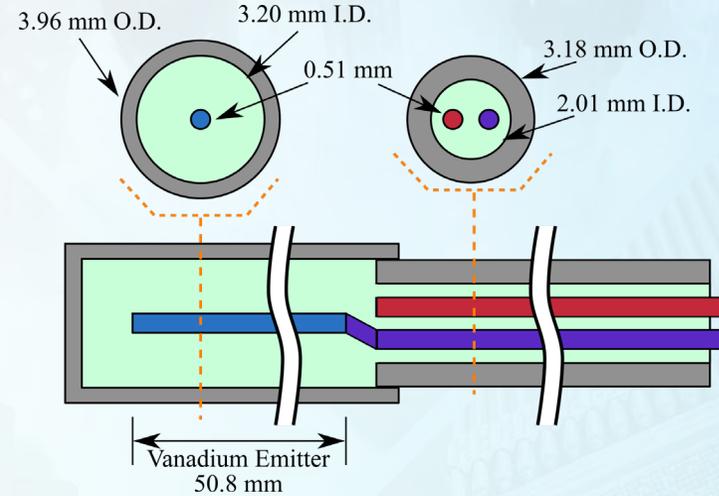
WIRE-21 measurement methods

	Temperature	Pressure	Flux/Fluence
<b>Active</b>	<ul style="list-style-type: none"> <li>• WEC Wireless</li> <li>• Thermocouple</li> <li>• Optical Fiber</li> </ul>	<ul style="list-style-type: none"> <li>• WEC Wireless</li> <li>• [Externally]</li> </ul>	<ul style="list-style-type: none"> <li>• Self-Powered Neutron Detector</li> </ul>
<b>Passive</b>	<ul style="list-style-type: none"> <li>• SiC Thermometry</li> </ul>		<ul style="list-style-type: none"> <li>• Activation Flux Wires</li> </ul>



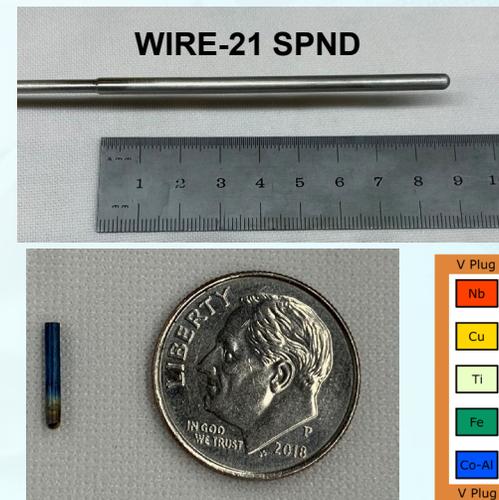
# SPND Devices

- Emitter: V ( $62.6 \pm 0.1$  mg)
- Insulation: MgO
- Collector: Inc600
- Leads: Inc600
- Positioned  $\pm 5, \pm 15$  cm above/below HFIR midplane

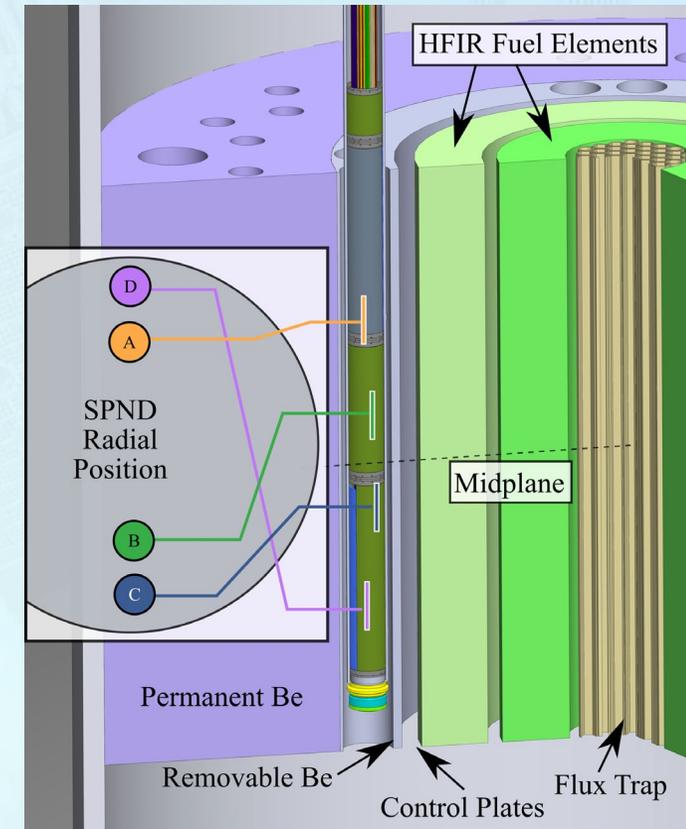


SPND Dimensional Comparison

	Radius emitter	Radius insulator	
WIRE-21	0.255	1.6	
Thermocoax	0.25	0.43	[3]
INL Small	0.24	0.575	[4]
INL Large	0.39	0.69	[4]



Flux activation wires



SPND Positions

[3] Vermeeren, et al., ANIMMA 2019, EPJ Web of Conferences **225**, 04015 (2020)

[4] Palmer, et al., Conceptual Design Report for the I2 Instrumentation Experiment in ATRC. INL/MIS-19-55710 (2019)

# SPND Theory

Current ( $I_0$ ) under steady state:

$$I_0 = k \times N \times \sigma_c \times \Phi \times e$$

Step increase in neutron flux:

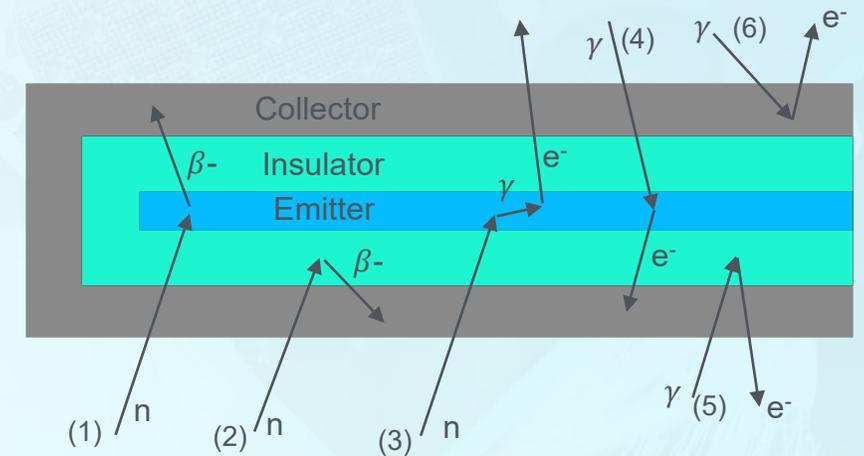
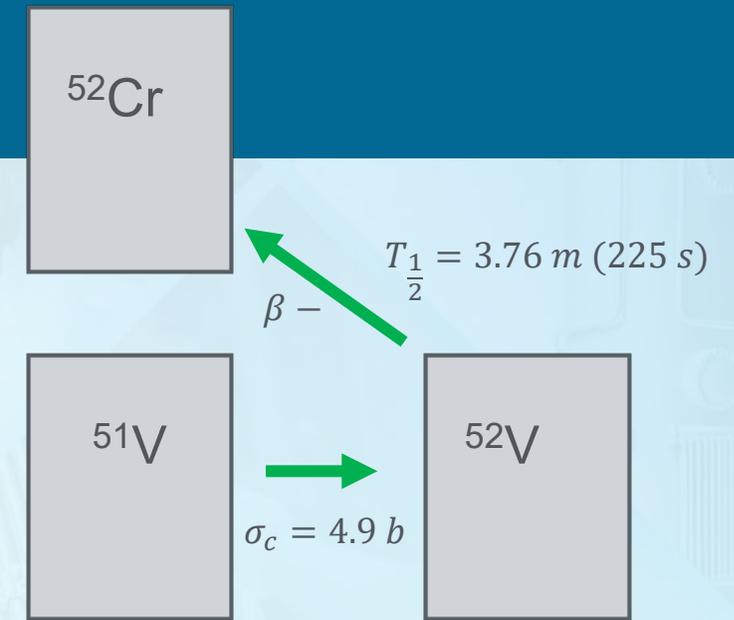
$$I(t) = I_0 \left( 1 - \exp\left(\frac{-\ln(2) \times t}{T_{1/2}}\right) \right)$$

Step decrease in neutron flux:

$$I(t) = I_0 \exp\left(\frac{-\ln(2) \times t}{T_{1/2}}\right)$$

Where

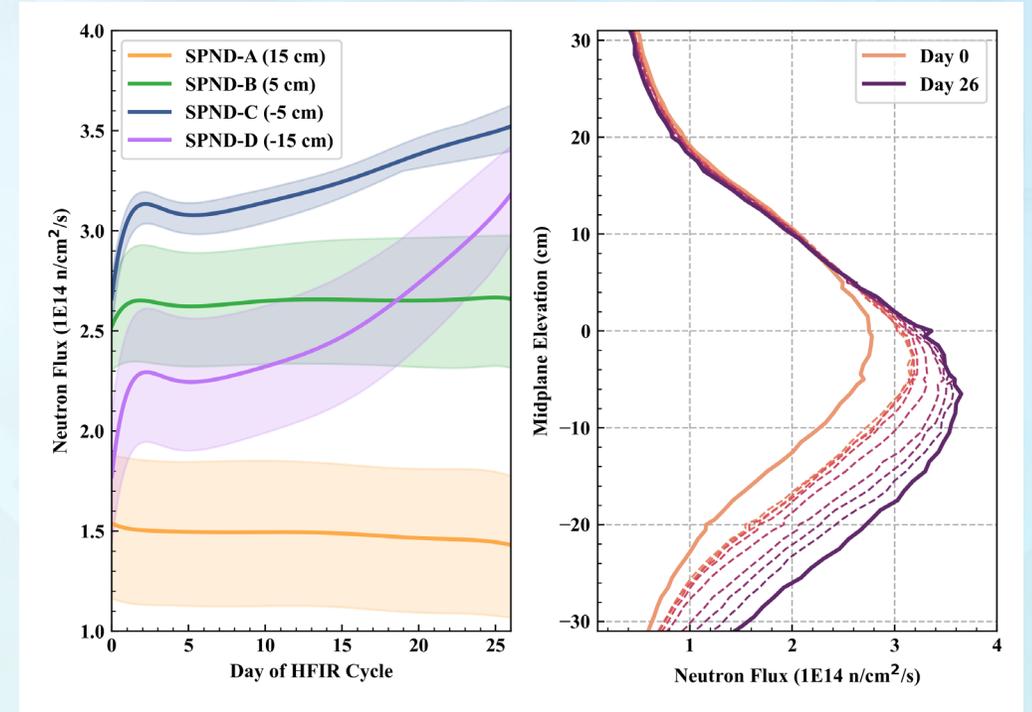
- $k$  = detection efficiency factor
- $N$  = number of useful target nuclei in emitter
- $\sigma_c$  = neutron capture cross section
- $\Phi$  = neutron flux
- $e$  = elementary charge
- $T_{1/2}$  = emitter isotope half-life



Adapted from Moreira & Lescano, Ann. Nucl. Energy 58 (2013) 90.

# Radiation Transport Modeling

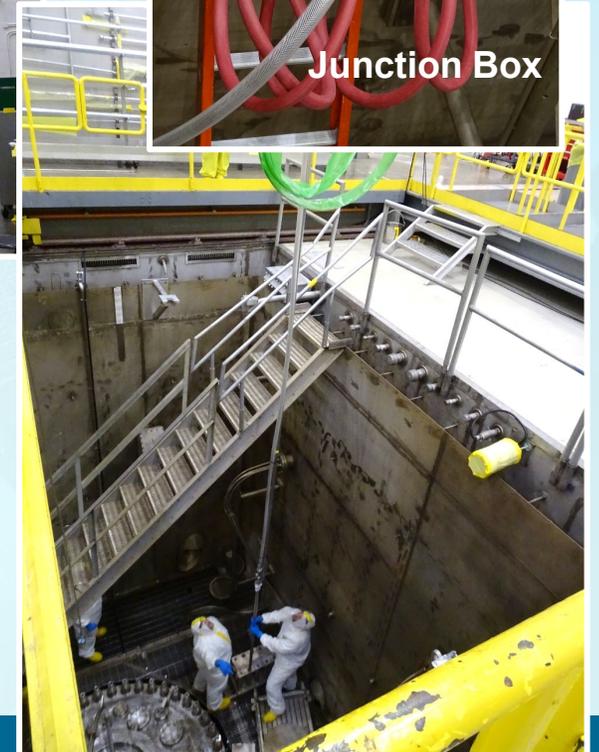
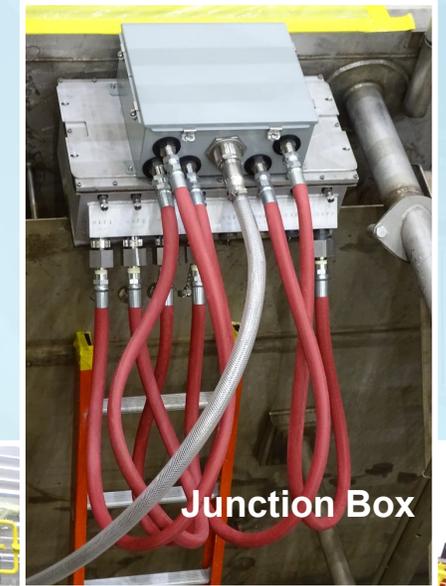
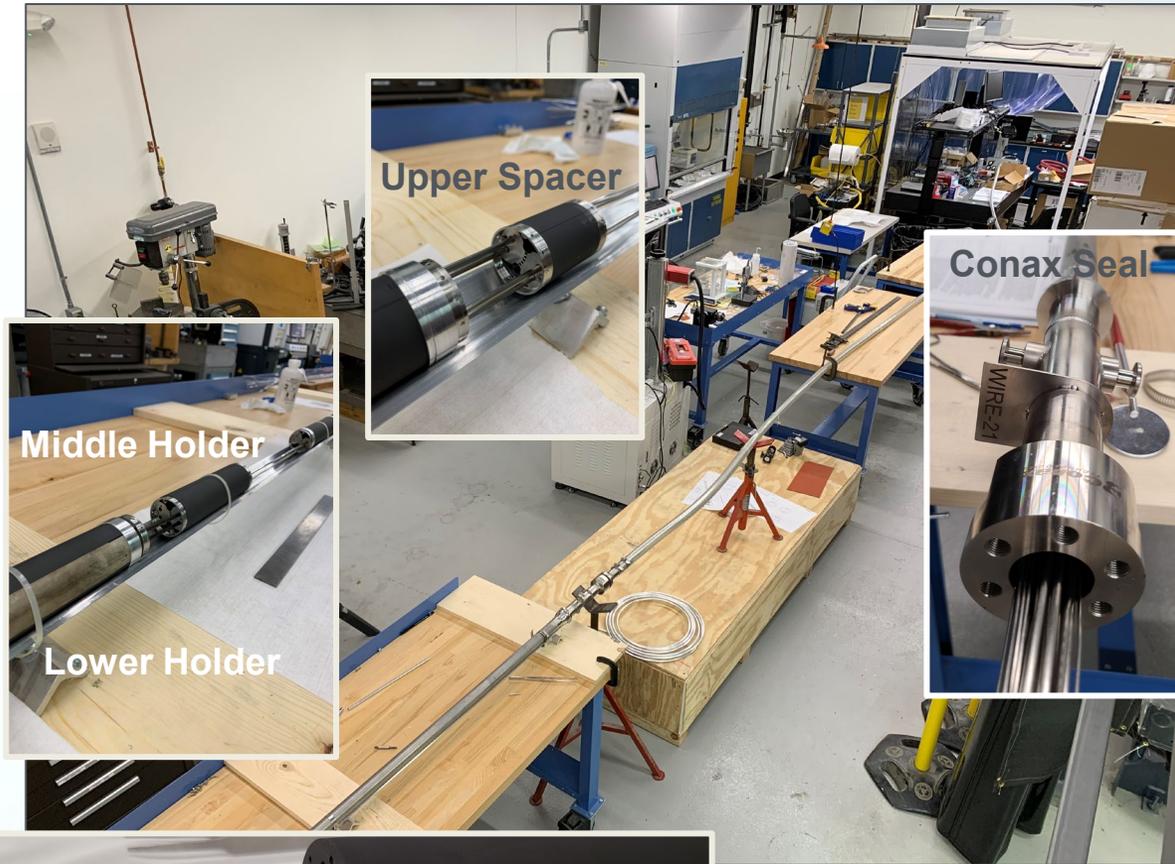
- Representative geometry and materials of WIRE-21 modeled using ORNL developed HFIRCON [5] code
  - Time-dependent, coupled radiation transport and depletion code
  - WIRE-21 geometry divided into 1 cm axial subsections
  - Modeled for 10 timesteps over one 26-day HFIR cycle
  - Cells separated into “core facing” and “reflector facing” to capture shielding effects
- HFIRCON model provides
  - 256 group neutron flux
  - Material heat generation rates
    - Prompt gamma, neutron
    - Fission product decay heat
    - Local activation/decay heat



(Left) Modeled time dependent, thermal neutron flux ( $E_n < 0.025$  eV) for each SPND. (Right) Spatial- and time-dependent thermal neutron flux across reflector.

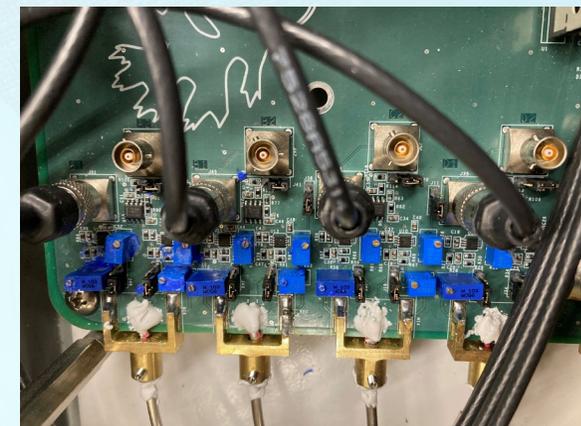
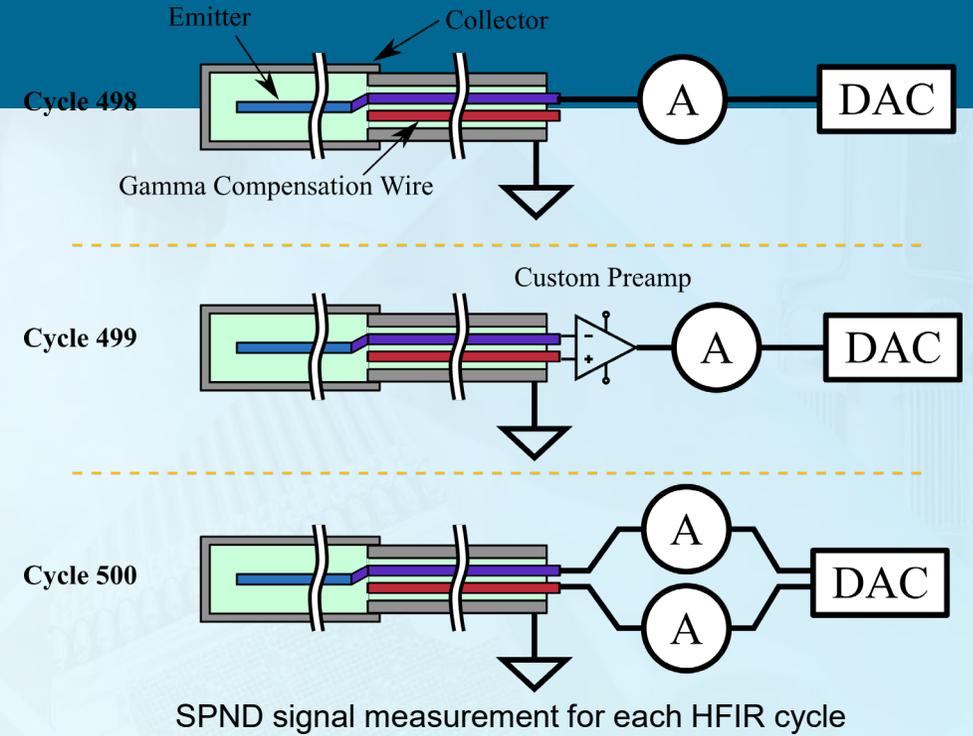
[5] C. Daily, et al., "HFIRCON Version 1.0.5 User Guide," ORNL/TM-2020/1742: Oak Ridge National Laboratory, Oak Ridge, TN (2020).

# WIRE-21 Assembly and Installation



# SPND Measurement Configuration

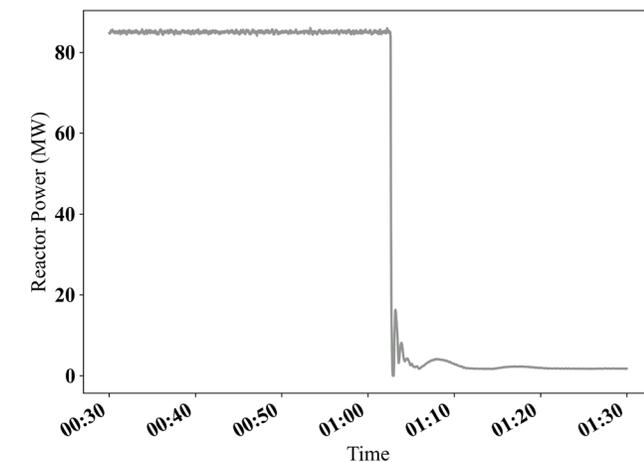
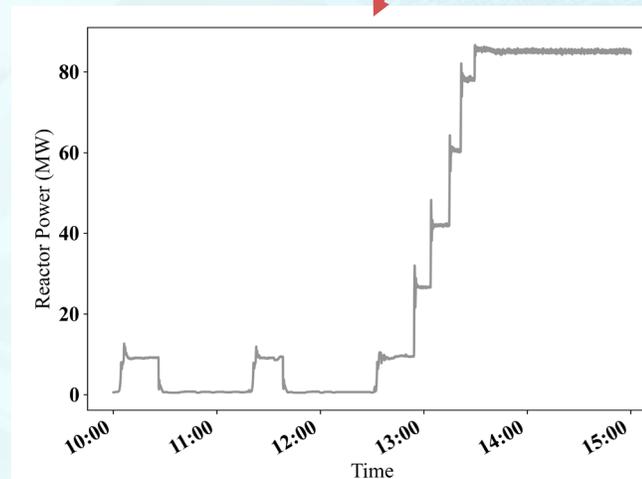
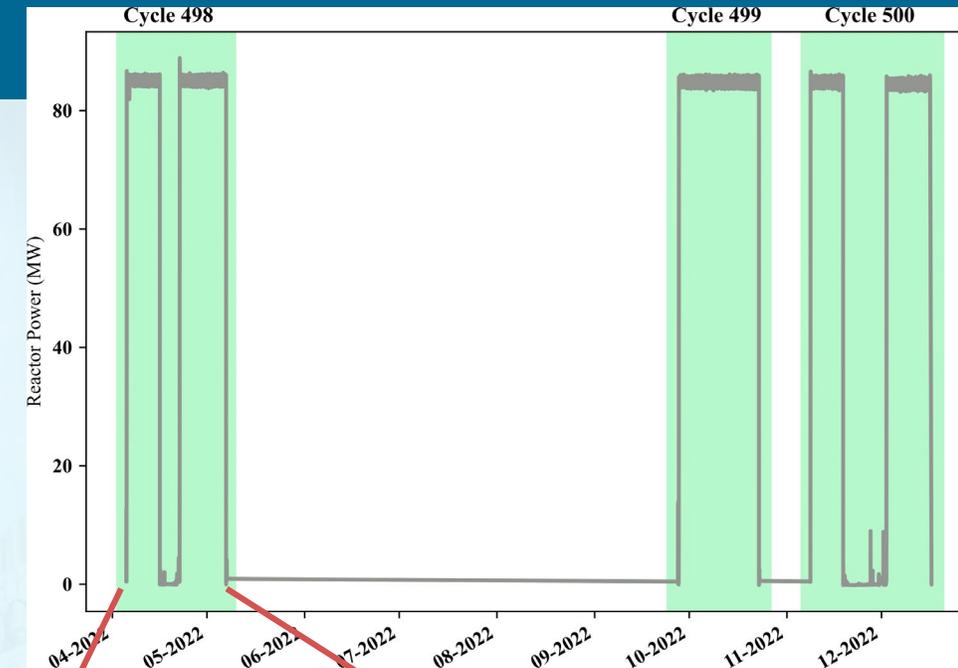
- Current measured using Keithley 6482 Dual Channel Picoammeter
- 3 different configurations for each cycle
- Cycle 498
  - x4 SPNDs, emitter wire only
- Cycle 499
  - x4 SPNDs, emitter/compensation differential signal
- Cycle 500
  - x2 SPNDs, emitter and compensation separately



Custom PCB for passthrough or differential measurement

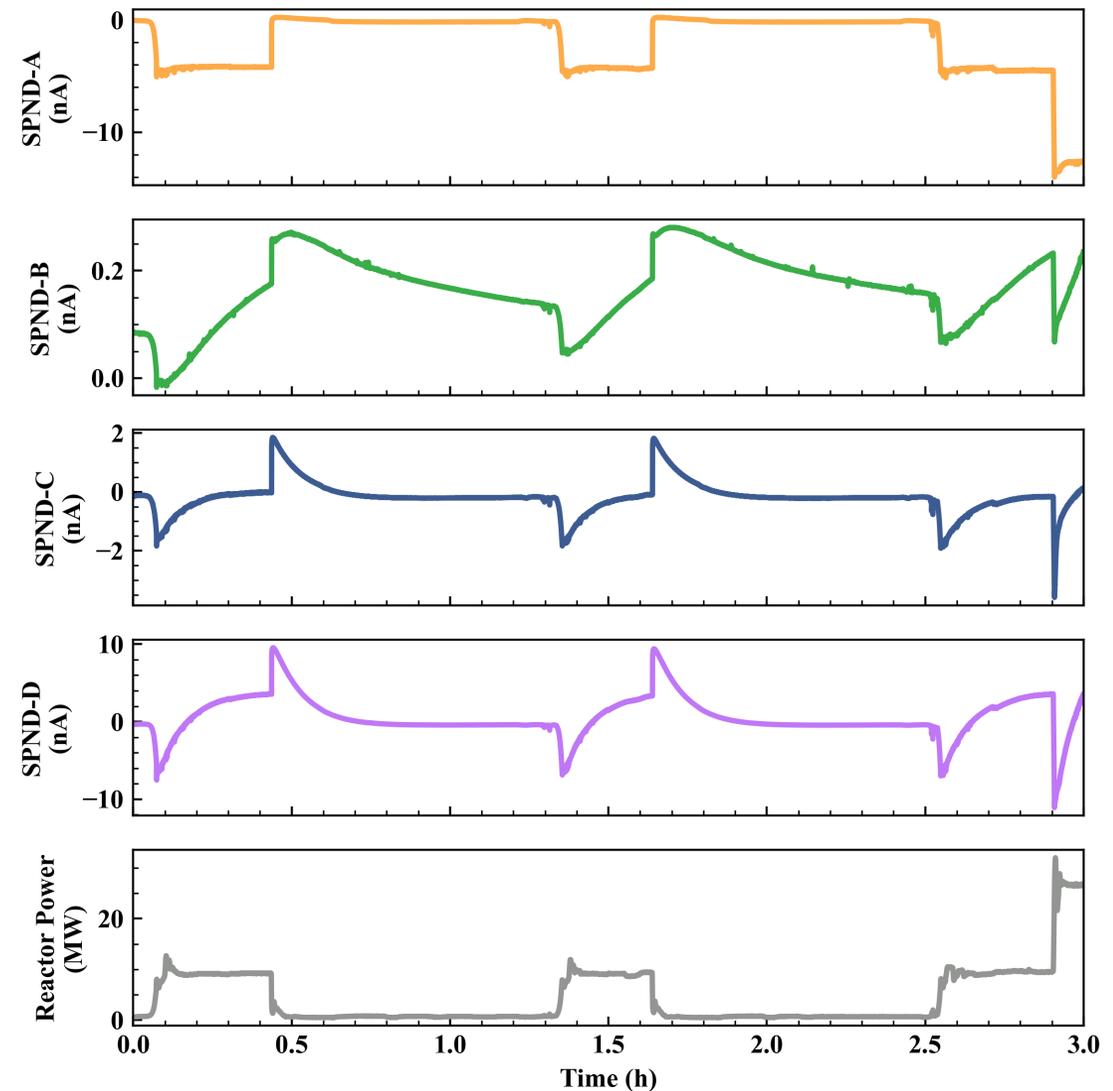
# WIRE-21 Operational History

- 3 HFIR cycles
  - 75 effective full power days (EFPDs)
  - $5.8 \times 10^{21}$  n/cm<sup>2</sup> thermal fluence
- Multiple temperature and pressure manipulations during reactor operation
- Startups and SCRAMs provided interesting transients for SPND investigation



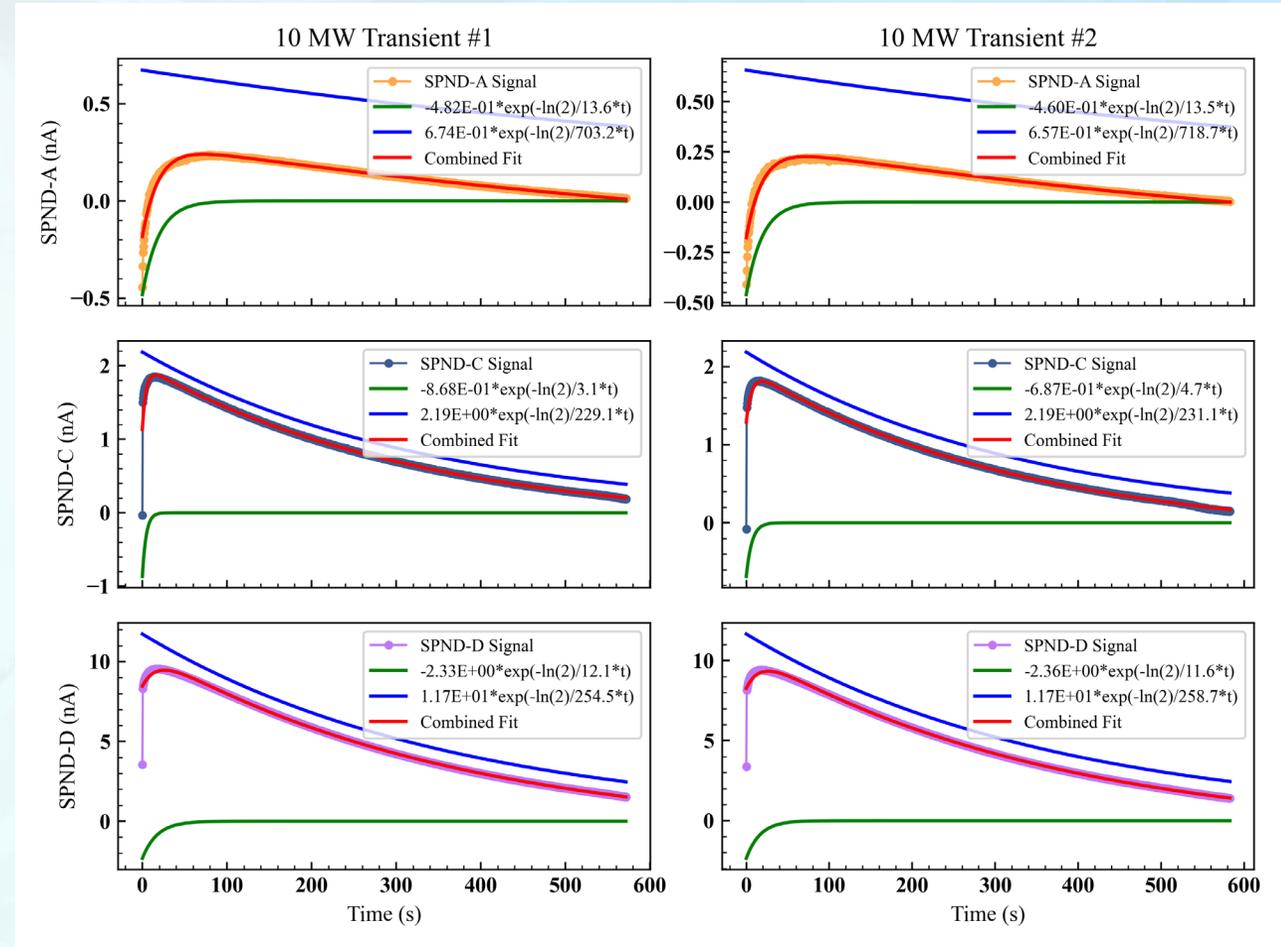
# Reactor Power Transients (498)

- Reactor power raised to 10 MW several times during startup
- SPND-A,-C,-D showed similar but unusual behavior
  - Prompt negative current
  - Exponential positive current
- SPND-B exhibited linear increase in current with steady power
  - Assumed SPND-B was either broken during installation or compensation wire was being measured



# Signal Curve Fitting

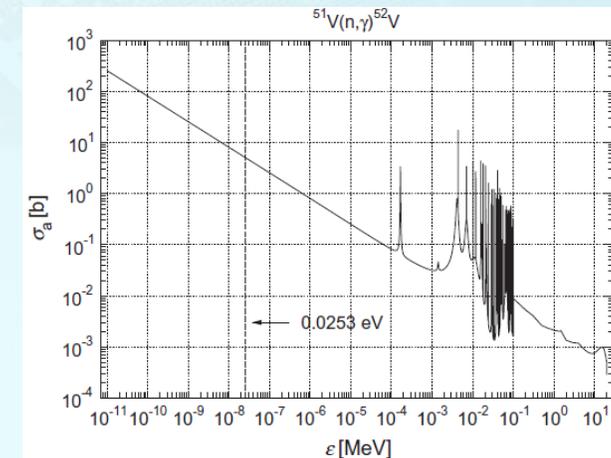
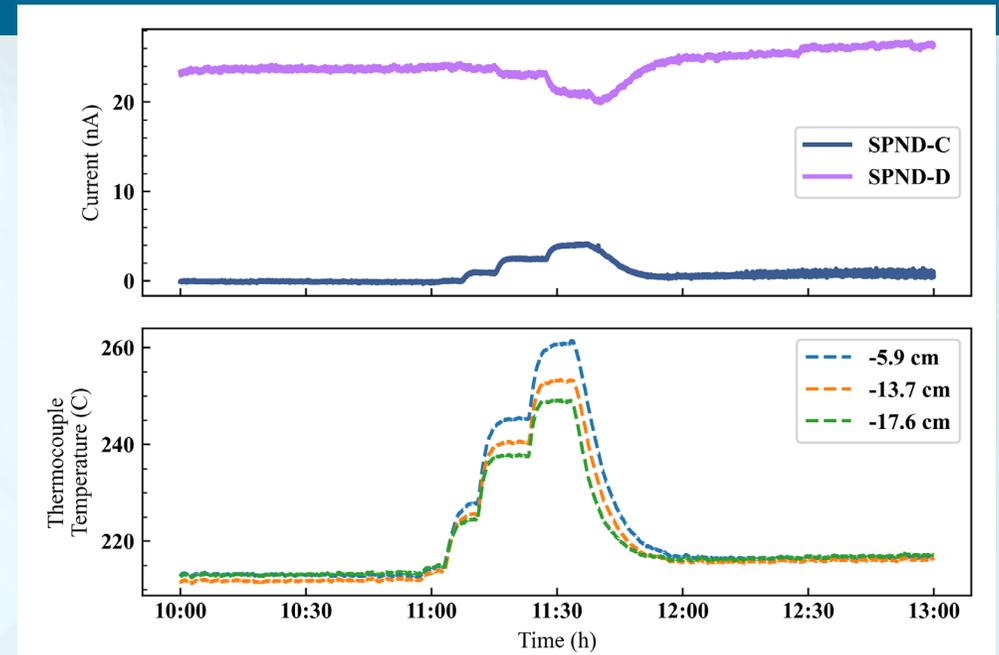
- SPND signal following power decrease was fit to equation of the form:
 
$$I(t) = A + B * \exp(p \times t) + C * \exp(q \times t)$$
- Exponential coefficients showed good agreement with  $T_{1/2} = 225$  s of  $^{52}\text{V}$  for -C/-D
  - Validates slow response signal is neutron capture in V
- Less conclusive for -A ( $T_{1/2} = 700$  s)
  - Likely caused by large difference in magnitude between  $\gamma$  and neutron signal
- Gamma component had very short (prompt)time constant (3-13 s) and negative contribution in all 3 SPNDs



Signal curve fitting for cycle 498 startup transient (0.4 and 1.6 hrs)

# SPND Temperature Response

- Experiment temperature was increased stepwise for WEC sensors during cycle 498
- SPND-C/-D signals followed temperature increase, though in different directions
- Doppler broadening should result in increased SPND signal
- Higher temperature could be perturbing neutron energy in  $1/v$  region



Moreira & Lescano, Ann. Nucl. Energy 58 (2013) 90.

# Concluding Remarks

- WIRE-21 experiment was performed in 2022 to test novel in-pile sensors
- Included the first SPND flux measurements in HFIR
- Three SPNDs demonstrated neutron induced signal
- Two SPNDs agreed with modeled flux trends
- SPNDs showed response to temperature
- Future SPNDs require optimization to mitigate gamma signal

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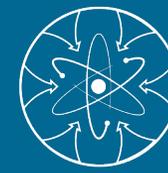
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**Thank You**